

of Transportation

Research and Special Programs Administration 400 Seventh St., S.W. Washington, D.C. 20590

COMPETENT AUTHORITY CERTIFICATION FOR A FISSILE

RADIOACTIVE MATERIALS PACKAGE DESIGN CERTIFICATE USA/6581/AF-85, REVISION 26

This certifies that the radioactive materials package design described below has been certified by the Competent Authority of the United States as meeting the regulatory requirements for a packaging for fissile radioactive materials as prescribed in the regulations of the International Atomic Energy Agency and the United States of America.

- 1. Package Identification Siemens Power Corporation No. 51032-1
- 2. <u>Packaging Description and Authorized Radioactive Contents</u> as described in U.S. Nuclear Regulatory Commission Certificate of Compliance No. 6581, Revision 33 (attached).

3. Criticality -

- a. Criticality Transport Index 0.4
- b. Allowable Number of Packages per Conveyance 250
- c. The criticality analysis considered both the presence and absence of water in all void spaces.

4. General Conditions -

- a. Each user of this certificate must have in his possession a copy of this certificate and all documents necessary to properly prepare the package for transportation.
- b. Each user of this certificate, other than the original petitioner, shall register his identity in writing to the Office of Hazardous Materials Technology, (DHM-23), Research and Special Programs Administration, U.S. Department of Transportation, Washington D.C. 20590-0001.
- c. This certificate does not relieve any consignor or carrier from compliance with any requirement of the Government of any country through or into which the package is to be transported.

^{1 &}quot;Safety Series No. 6, Regulations for the Safe Transport of Radioactive Materials, 1985 Edition (As amended 1990)", published by the International Atomic Energy Agency (IAEA), Vienna, Austria.

² Title 49, Code of Federal Regulations, Parts 100 - 199, United States of America.

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- d. Records of Quality Assurance activities required by Paragraph 209 of the IAEA regulations shall be maintained and made available to the authorized officials for at least three years after the last shipment authorized by this certificate. Consignors and consignees in the United States exporting or importing shipments under this certificate shall satisfy the requirements of Subpart H of 10 CFR 71.
- 5. Marking and Labeling The package shall bear the marking USA/6581/AF-85 in addition to other required markings and labeling.
- 6. Expiration Date This certificate expires on May 31, 2009.

This certificate is issued in accordance with paragraph 712 of the IAEA Regulations and Section 173.471 of Title 49 of the Code of Federal Regulations, in response to the petition and information dated January 7, 2004 submitted by Framatome ANP, Richland, WA, and in consideration of other information on file in this Office.

Robert A. McGuile
Associate Administrator for
Hazardous Materials Safety

FEB - 9 2004

(DATE)

Revision 26 - Issued to incorporate NRC Certificate of Compliance No. 6581, Revision 33, and to extend the expiration date.

NRC FORM 618 U.S. NUCLEAR REGULATORY COMMISS (8-2000) 10 CFR 71 CERTIFICATE OF COMPLIANCE										
<u> </u>	FOR RADIOACTIVE MATERIAL PACKAGES									
	a. CERTIFICATE NUMBER 6581	b. REVISION NUMBER	c. DOCKET NUMBER 71-6581	d. PACKAGE IDENTIFICATION NUMBER USA/6581/AF	PAGE 1	OF	PAGES 5			

2. PREAMBLE

- a. This certificate is issued to certify that the package (packaging and contents) described in Item 5 below meets the applicable safety standards set forth in Title 10, Code of Federal Regulations, Part 71, "Packaging and Transportation of Radioactive Material."
- b. This certificate does not relieve the consignor from compliance with any requirement of the regulations of the U.S. Department of Transportation or other applicable regulatory agencies, including the government of any country through or into which the package will be transported.
- 3. THIS CERTIFICATE IS ISSUED ON THE BASIS OF A SAFETY ANALYSIS REPORT OF THE PACKAGE DESIGN OR APPLICATION

a. ISSUED TO (Name and Address)
 Framatome ANP, Inc.
 2101 Horn Rapids Road
 Richland, WA 99352-0130

b. TITLE AND IDENTIFICATION OF REPORT OR APPLICATION

Framatome ANP, Incorporated Consolidated License Application dated January 20, 2003, as supplemented.

4. CONDITIONS

This certificate is conditional upon falling the requirements of 10 CFR Part 71, as applicable, and the conditions specified below.

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- (a) Packaging
 - (1) Model No. \$1032
 - (2) Description

A steel shipping container for tuel bundles, consisting of a strongback and fuel bundle clamping assembly, shock mounted to a steel pure container. Steel separator blocks are bolted between fuel assemblies. The separator blocks are a minimum of these wide by approximately 8 inches high and 9 inches long, with a minimum nominal 3/8-inch thick wall. The outer container is approximately 43 inches in diameter by 216 inches long. The maximum weight of the package, including contents, is 7,500 poures.

(3) Drawings

The packaging is constructed and assembled in accordance with the following Siemens Power Corporation Drawing Nos.:

EMF-309,813, Rev. 2, Sheets 1 and 2

EMF-303,359, Rev. 7

EMF-303,360, Rev. 6

EMF-303,898, Rev. 5

EMF-300,607, Rev. 3

EMF-309,582, Rev. 0

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5.(b) Contents

(1) Type and Form of material

(i) Unirradiated fuel rods consisting of uranium dioxide fuel pellets clad in zirconium alloy or stainless steel tubes. Fuel rods must be in one of the following configurations:

						<u>T15x15</u>	
					5.	<u>Square</u>	<u>T15 x15</u>
	<u>Type</u>	15x15	17x171	GENE	Rod Container ³	<u>Array</u> <u>Assemblies⁴</u>	<u>Cruciform</u> <u>Assemblies</u>
	Maximum Enrichment (%U-235)	5.0	5.0	5.0	5.0	5.0	2.8
	Rods Per Assembly	204	264	any number	any number	208	28
	Nominal Rod Pitch (in.)	0.563	0.496	NA	NA	0.527	0.556
	Maximum Pellet Density (%TD)	95	95	95	95	95	95
	Maximum Clad OD (in.)	0.430	ø.380	0.500	0.500	0.400	0.500
	Minimum Clad OD (in.)	0.410	0.855	···· 0:260 []	0.260	0.364	0.260
	Minimum sum of clad the ness and pellet-clad gap ⁵ (in.)	2028	0.023	0.023	0.023	0.016	0.023
	Assembly Cross Section (in)	THE N	8.432	8,25	NA	7.91	8.25
	Active Fuel Length (in.)	196	196	196	196	196	116
	Fuel Rod Arrangement (Figure Number in Application)	71.1	11.2	NA	NA	VII-1	VII-3
-	Table Notes		次文	TXT			

Table Notes

Fuel assemblies consisting of a maximum 264 fuel rods in a 17x17 square array with any number of edge rods missing.

Fuel assemblies consisting of any number of fuel rods in a square array with a maximum assembly cross section of 8.25 inches square.

Any number of fuel rods positioned in a rod container. The rod container consists of a schedule 40 steel pipe with a maximum nominal diameter of 5 inches.

Fuel assemblies consisting of a maximum of 208 fuel rods in a 15x15 square array, with any number of edge rods missing.

Minimum sum of the cladding wall thickness and the pellet-clad radial gap, ((Min Clad OD - Max Pellet OD)/2), in.

NRC F (8-2000) 10 CFR 7			ATE OF COMPL FIVE MATERIAL F		ULATORY	СОММ	ISSION
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5.(b) Contents (Continued)

(ii) Unirradiated fuel assemblies, composed of uranium dioxide fuel pellets clad in zirconium alloy or stainless steel tubes. Uranium is enriched to a maximum of 5.05 wt% in the U-235 isotope. The fuel assemblies may contain inserted control rod assemblies. The fuel assemblies have the following specifications:

<u>Type</u>	<u>L1</u>	<u>L2</u>	<u>L4</u>
Array Size	Q Q 5×15	15x15	17x17
Fueled Rods Per Assembly	EAR REEGU	208	264
Minimum No. of Non-Fueled Rods	17	17	25
Nominal Rod Pitch (in.)	0.568	0.568	0.496
Maximum Pellet Diameter (n)	0.3707	0.3742	0.3232
Maximum Pellet Density (%TD)	97.5	9 7:5	97.5
Nominal Clad OD (in.)	0.430	0.430	0.374
Minimum sum of clad thickness and pellet-clad gap ¹ (in.)	11114028	0.023	0.023
Assembly Cross Section (m)		8.52	8.432
Active Fuel Length (in.)	196	196	196
Fuel Rod Arrangement (Figure Number in Application)	VIII-1	VIII-1	VIII-2
Table Notes:	XXXXX	-	

Minimum sum of the cladding wall thickness and the pellet-clad radial gap, ((Min Clad OD - Max Pellet OD)/2), in.

(2) Maximum quantity of material per package

Maximum quantity of material within a package may not exceed a Type A quantity. Total weight of fuel assemblies, or fuel rods, and rod containers, not to exceed 3400 pounds, and

(i) For the contents described in 5(b)(1)(i), the total weight of fuel assemblies:

Two full length fuel assemblies. Two short fuel assemblies may be substituted for each full length fuel assembly provided the two short assemblies are shipped end-to-end and the total fuel length does not exceed the maximum fuel length for a full length assembly; or

Two rod containers.

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- 5.(b) Contents (Continued)
 - (ii) For the contents described in 5(b)(1)(ii):

Two fuel assemblies.

(c) Transport Index for Criticality Control (Criticality Safety Index)

Minimum transport index to be shown on label for nuclear criticality control:

0.4

- 6. Each fuel assembly must be unsheathed or must be enclosed in an unsealed polyethylene sheath which will not extend beyond the encis of the fuel assemblies. The ends of the sheaths must not be folded or taped in any manner that would prevent the flow of liquids into or out of the sheathed fuel assemblies.
- 7. Hydrogenous shims are not permitted within the fuel assemblies.
- 8. Separator blocks, shock mounts and fuel element clamp assemblies must be in accordance with Tables 2.2, 2.3, 2.4, 2.5, and VII-3 of the abblication
- 9. Each separator block must be attached to the strongback by one of the following methods, as shown in Drawing No. EMF-809,813, Rev. 2, Shoot 2:
 - (a) Two, 5/8-11 UNC Grade 6 steel cap screws and nuts. A 3/8-11 UNC Grade 2 (or better) steel stud may be substituted for page 1 the cap screws.
 - (b) Two, 1-8 UNC Grade 8 steel cap screws and nuts of 1-8 UNC Grade 8 steel stud may be substituted for one of the cap screws
- 10. The fuel assembly cross section is defined as the rod pitch times the number of rods on the edge of the assembly.
- 11. Rods containing gadolinia or other neutron poison are authorized but not required.
- 12. In addition to the requirements of Subpart G of 10 CFR Part 71:
 - (a) The package shall be prepared for shipment and operated in accordance with the procedures in Chapter 3.0 of the application.
 - (b) Each packaging shall be maintained in accordance with the procedures in Section 3.4 of the application.
 - (c) Each packaging shall meet the acceptance tests in Chapter 4.0 of the application.
 - (d) Each fuel rod shall be welded closed and certified to be leak-tight prior to shipment.

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- 13. The package authorized by this certificate is hereby approved for use under the general license provisions of 10 CFR §71.12.
- 14. Expiration date: May 31, 2009.

REFERENCES

Framatome ANP, Incorporated consolidated application dated January 20, and its supplements, May 8, June 18, July 7, and November 26, 2003.

John D. Monninger, Chie Licensing Section pent Fu@iProject Office of Nuclear Materials Safety and Safeguards

Date: December 30, 2003